

Case study: Plasma – Materials Interactions

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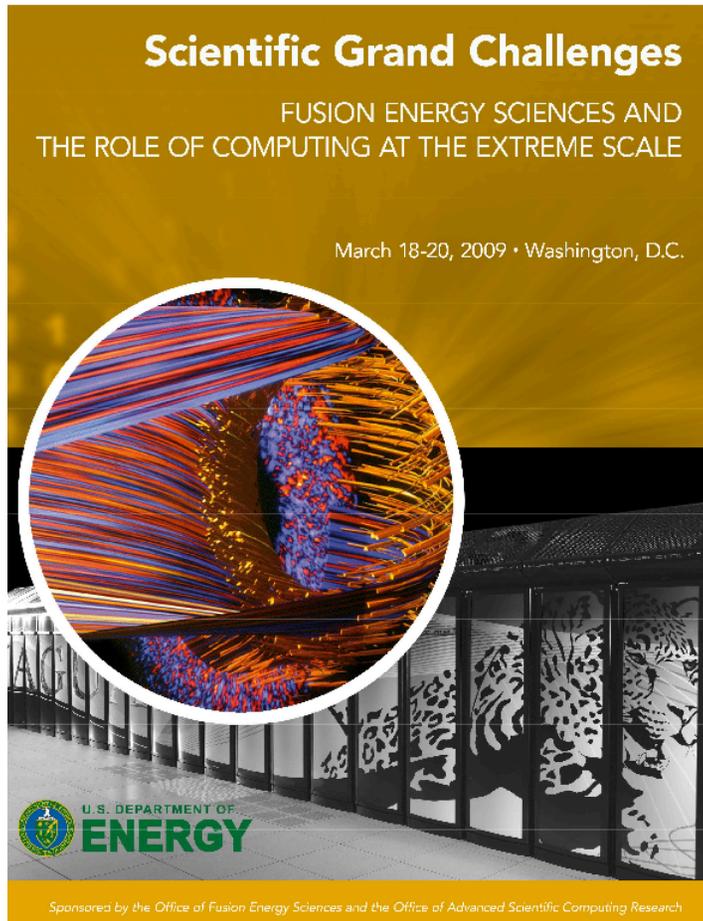
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Presentation Overview

- **Challenges of plasma facing components & structural materials in the (magnetic) fusion environment**
- **Multiscale phenomena governing materials changes & performance, and the multiscale modeling approach**
 - **Still very much a work in progress – no single, integrated code**
 - **Much of the modeling is performed on smaller, individual PI clusters**
- **Select set of codes, results and computing requirements**
- **Summary and future work**

Fusion Materials Challenges



Co-chaired by Bill Tang and David Keyes

Plasma Materials Interaction Science Challenges:

Modeling the edge and scrape-off layer plasmas. This includes modeling of turbulent transport and full coupling of plasma ions and electrons, neutrals, photons, and electromagnetic fields. In addition, plasma contamination from near-surface transport of sputtered or vaporized material and quantification of plasma facing component particle and photon fluxes (with predictions of instability regimes) should be considered.

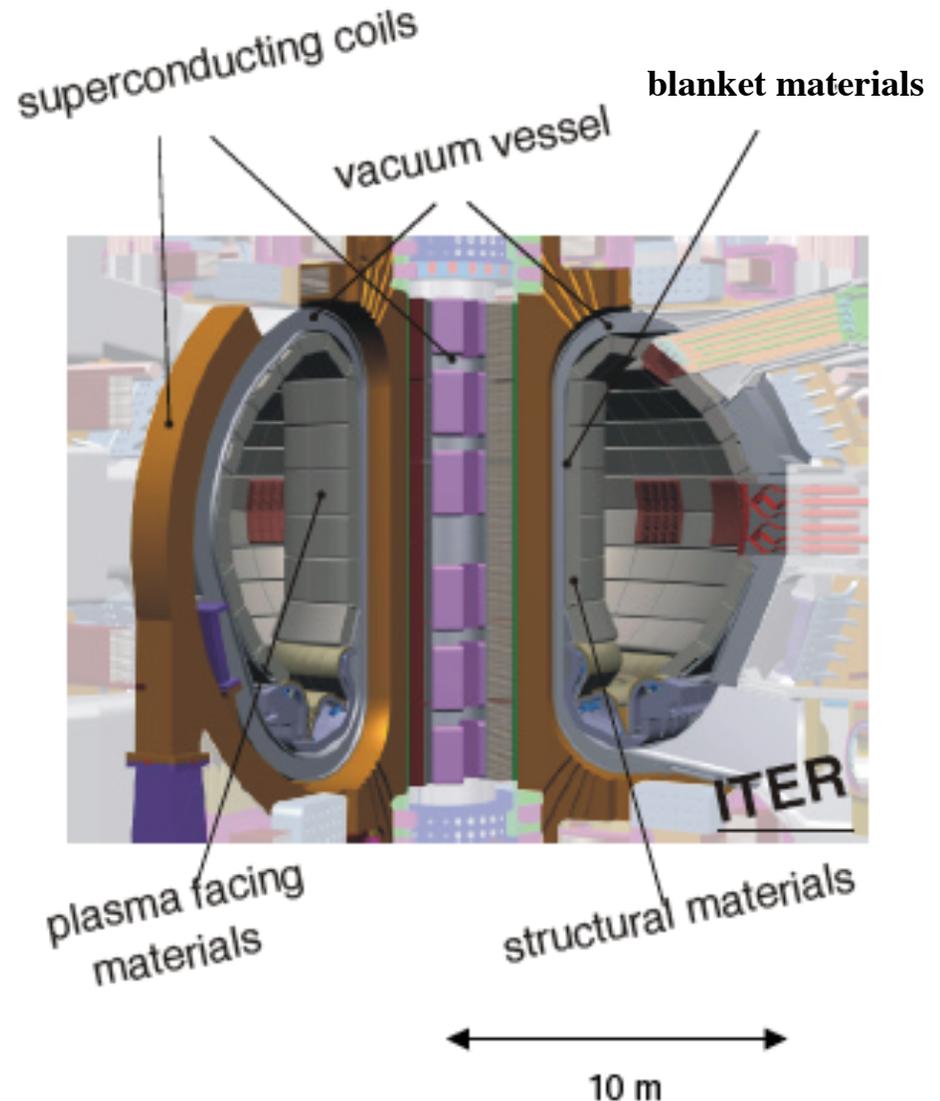
Predicting the near-surface material response to the extreme plasma fluxes of photons and particles under normal and transient operation. This includes predicting sputtering erosion/re-deposition and other time-integrated plasma facing component processes (e.g., dust formation and transport; helium- or deuterium-tritium-induced microstructure formation and flaking) and the resultant impurity transport, core plasma contamination, mixed-material formation, and tritium co-deposition in redeposited materials. The material and edge plasma response to transient processes such as high-powered edge localized modes vertical displacement events, plasma disruptions, and runaway electrons represent an important component of this effort.

Modeling the underlying structural materials response. This involves understanding the fundamental microstructure evolution and performance limits of structural materials in the fusion radiation environment that involve extreme cyclic thermo-mechanical stresses and simultaneous intense fusion neutron bombardment.

An overarching grand challenge will involve efficient integration of these to develop a comprehensive model.

*Materials issues in Magnetic Fusion Energy (ITER/DEMO)**

- **Magnetic fusion energy presents many materials challenges, including:**
 - **High thermal heat fluxes**
 - **Sputtering/blistering of plasma facing components**
 - **Radiation damage**
 - **Low induced radioactivity**
 - **Chemical compatibility**
 - **Joining/Welding**



*Ref: H. Bolt, Max-Planck Institute for Plasma Physics, Garching, Germany

Plasma Facing Components/Materials (ITER)*

Key issues

- erosion lifetime and plasma compatibility
- tritium inventory
- thermal transients
- **He blistering**
- heat removal:
- fabrication technology:
- neutron damage:

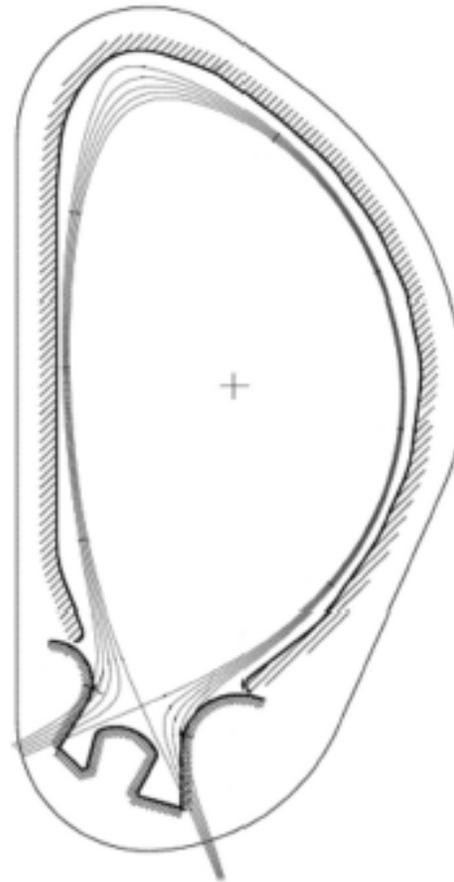
Leading candidate materials

PFC and Divertor:

- Be, W, C

Structural components:

- Fe-Cr steels, V-Cr-Ti, SiC



bulk plasma:

impurity tolerance

$W < 2 \cdot 10^{-5}$, reactor $< 10^{-4}$

Be, C: 10^{-2}

first wall:

modest flux of high energy neutral particles (100s eV),

low energy ions

divertor target:

high heat flux 10 (20) MW/m²

transient heat loads:

e.g. ELMs, disruptions

PFC Materials: Erosion & Blistering of C, Be & W*

- Neutron irradiation of C leads to decreases in thermal conductivity

First wall:

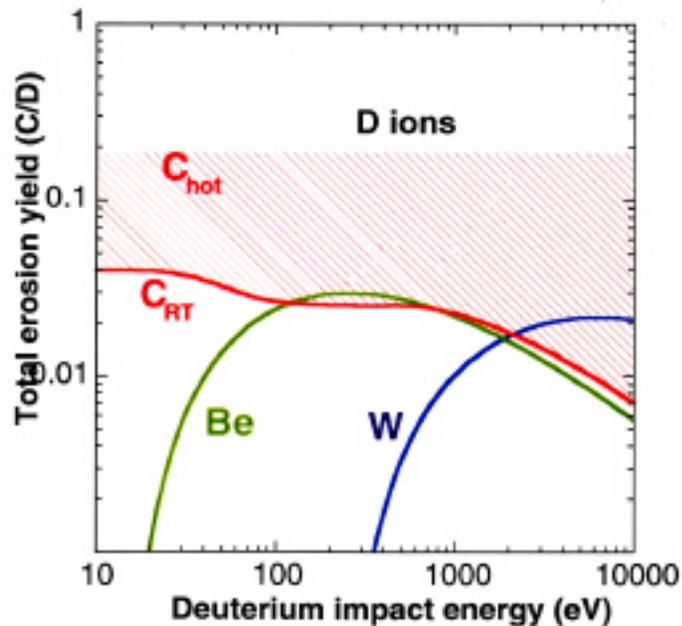
Erosion of

low Z materials

order of 3 mm/burn year: 15 mm in 5 years

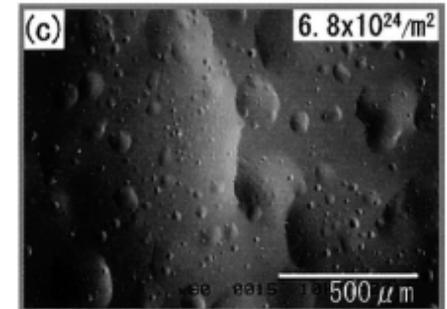
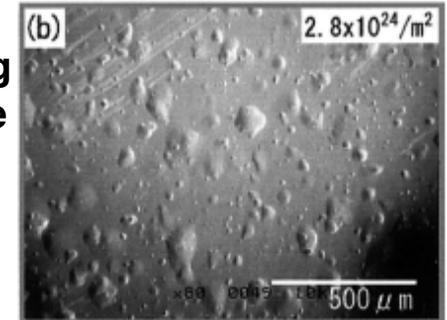
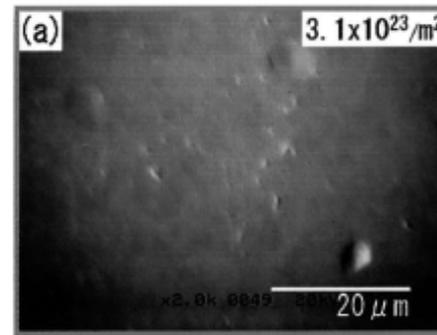
tungsten

order of 0.1 mm/burn year: 0.5 mm in 5 years



Blistering**:

α - (He) ion irradiation of PFC leads to blistering by growth of sub-surface He bubbles



Divertor:

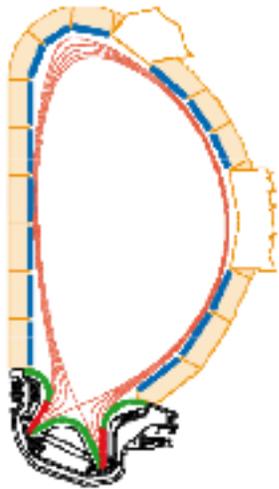
mostly redeposition of eroded wall material

*Presently: order of magnitude-knowledge;
data from ITER needed to assess PFM thickness*

Ref: * H. Bolt, Max-Planck Institute for Plasma Physics, Garching, Germany

** T. Shimada, Y. Ueda, M. Nishikawa, *Fusion Eng. & Des.* **66-68** (2003) 247.

*PFC Materials: Surface chemistry evolves as well**



ITER

- First wall on ITER
 - carbon 55 m²
 - tungsten 140 m²
 - beryllium 690 m²
- DEMO first wall / divertor
 - oxidation-resistant W alloys (e.g. W—Si—Cr)
- Variable local conditions (temperature, fluence, species...)
- Erosion and redeposition, impurities:
 - mixed phases (e.g. carbides, oxides, alloys)
- Layers on metals influence:
 - hydrogen inventory: reaction, diffusion, desorption
 - physical and chemical processes: sputtering, reactions
- Goal: qualitative and quantitative description of fundamental processes
 - formation and erosion of multi-component layers
 - influence of layers on hydrogen inventory
- **Include surface reactions in global integrated PWI model**

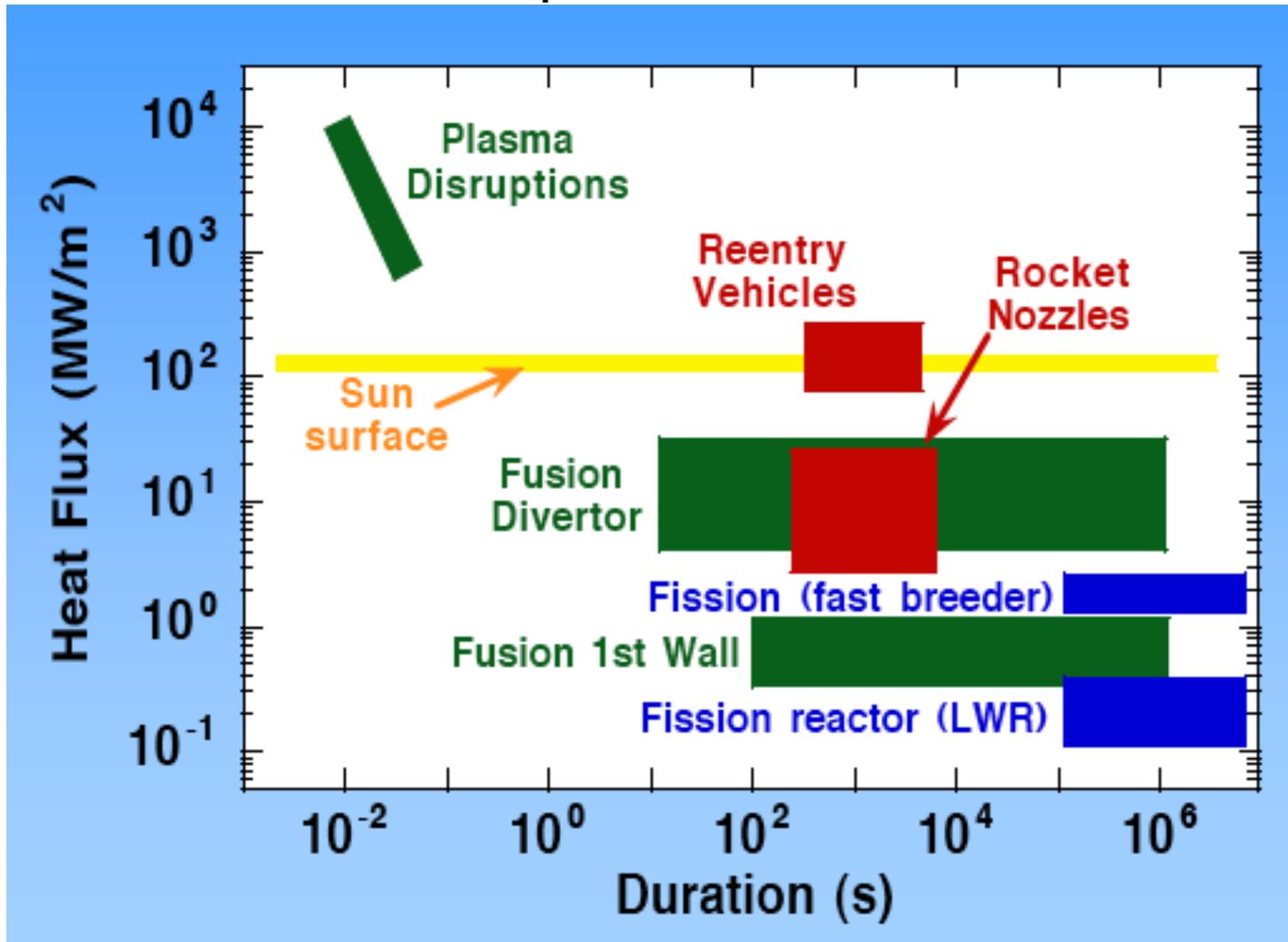
*PSI extrapolation challenges **

| Issue / Parameter | Present Tokamaks | ITER | DEMO | Consequences |
|---|-------------------------|-------------|-------------|---|
| Quiescent energy exhaust <i>GJ / day</i> | ~ 10 | 3,000 | 60,000 | - active cooling - max. tile thickness ~ 10 mm |
| Transient energy exhaust from plasma instabilities $\Delta T \sim MJ / A_{wall}(m^2) / (1 ms)^{1/2}$ | ~ 2 | 15 | 60 | - require high $T_{melt/ablate}$ - limit? ~ 60 for C and W - surface distortion |
| Yearly neutron damage in plasma-facing materials <i>displacements per atom</i> | ~ 0 | ~ 0.5 | 20 | - evolving material properties: thermal conductivity & swelling |
| Max. gross material removal rate with 1% erosion yield <i>(mm / operational-year)</i> | < 1 | 300 | 3000 | - must redeposit locally - limits lifetime - produces films |
| Tritium consumption <i>(g / day)</i> | < 0.02 | 20 | 1000 | - Tritium retention in materials and recovery |

* D. Whyte, IHHFC, Dec 2008

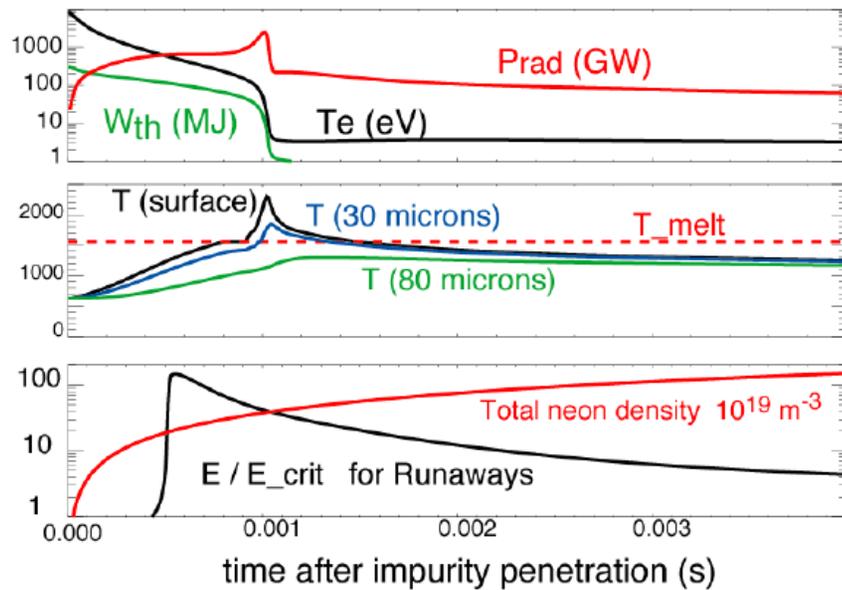
*Fusion materials challenges: Heat flux**

Comparison of Heat fluxes

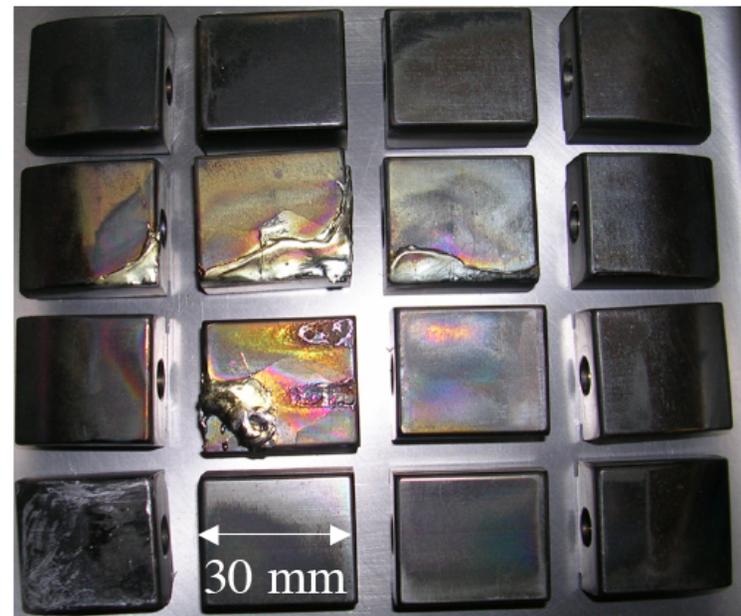


* S.J. Zinkle

High thermal heat fluxes – transients evolve over ms*



C-Mod Molybdenum ($T_{melt}=2900\text{ K}$) limiter melted during disruptions



- Dilute MFE plasma ($n \sim 10^{20}\text{ m}^{-3}$) extinguished by small particulate
 - 2 mm “drop” of W == $N_{e,ITER}$

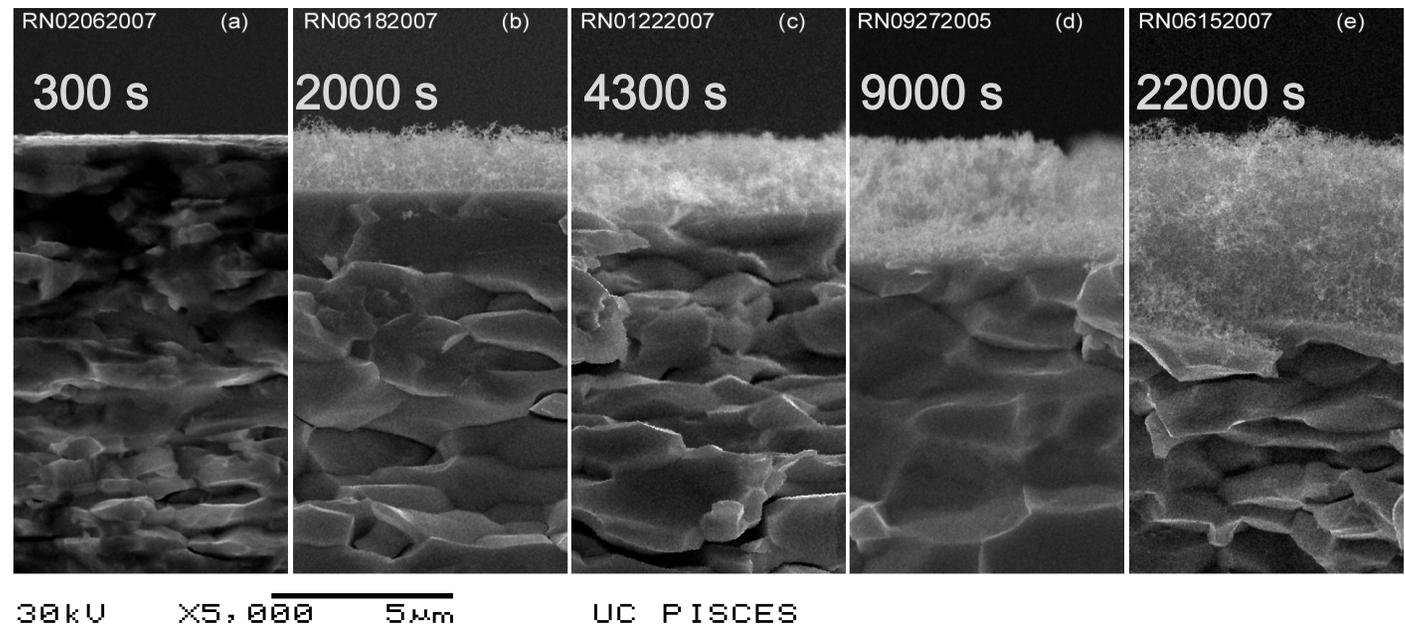
* D. Whyte, IHHFC, Dec 2008

Combined thermal and particle fluxes*

Dust formation in ITER PFC mix from several possible sources*:

- Deposited layer disintegration under transient loads → most likely in divertor were layers most likely to grow
- He-induced nano-morphology → dust formation in steady state, enhanced “non-atomistic” erosion rates on W

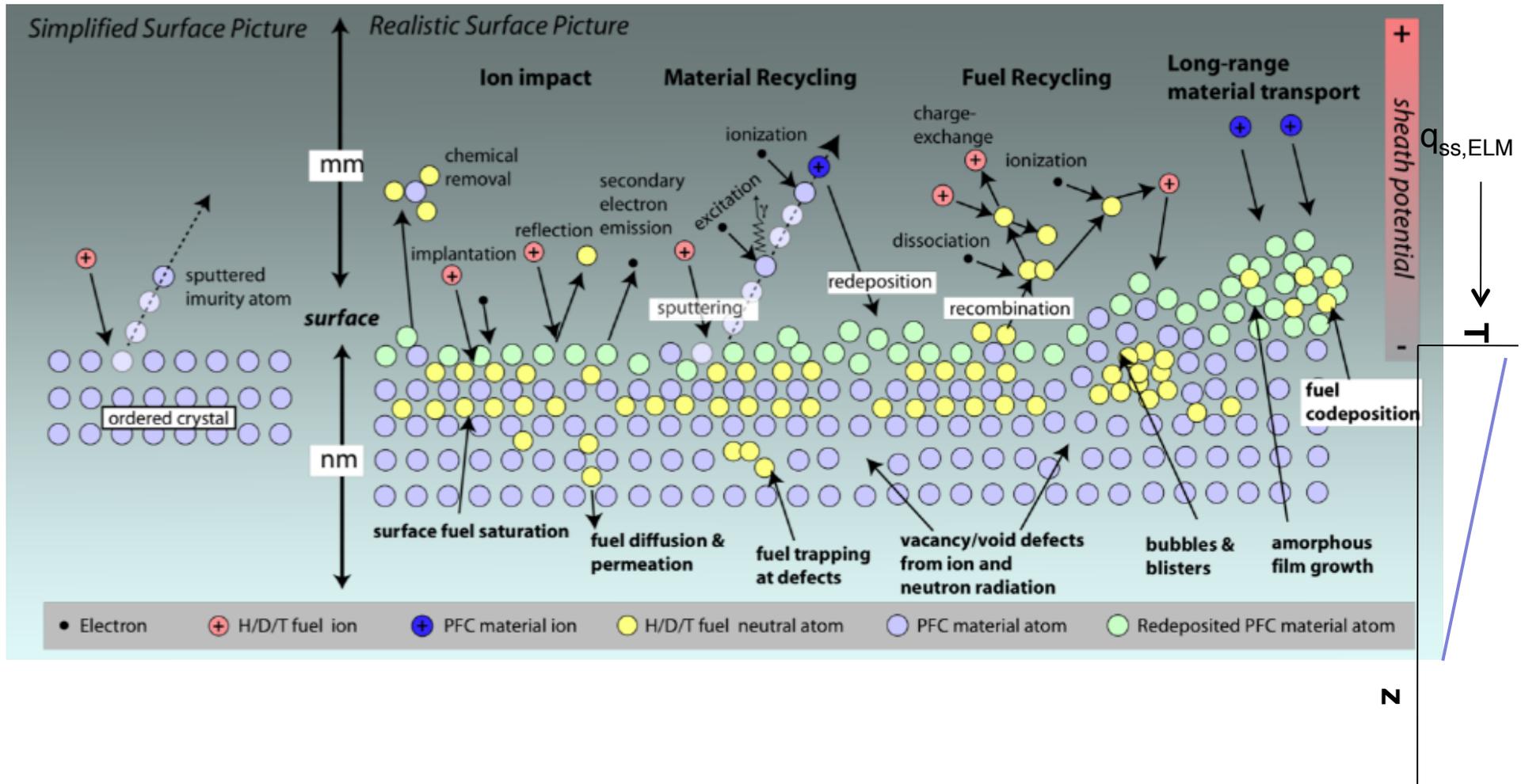
M. J. Baldwin et al., PSI 2008



$$T_s = 1120 \text{ K}, \Gamma_{\text{He}^+} = 4\text{--}6 \times 10^{22} \text{ m}^{-2}\text{s}^{-1}, E_{\text{ion}} \sim 60 \text{ eV}$$

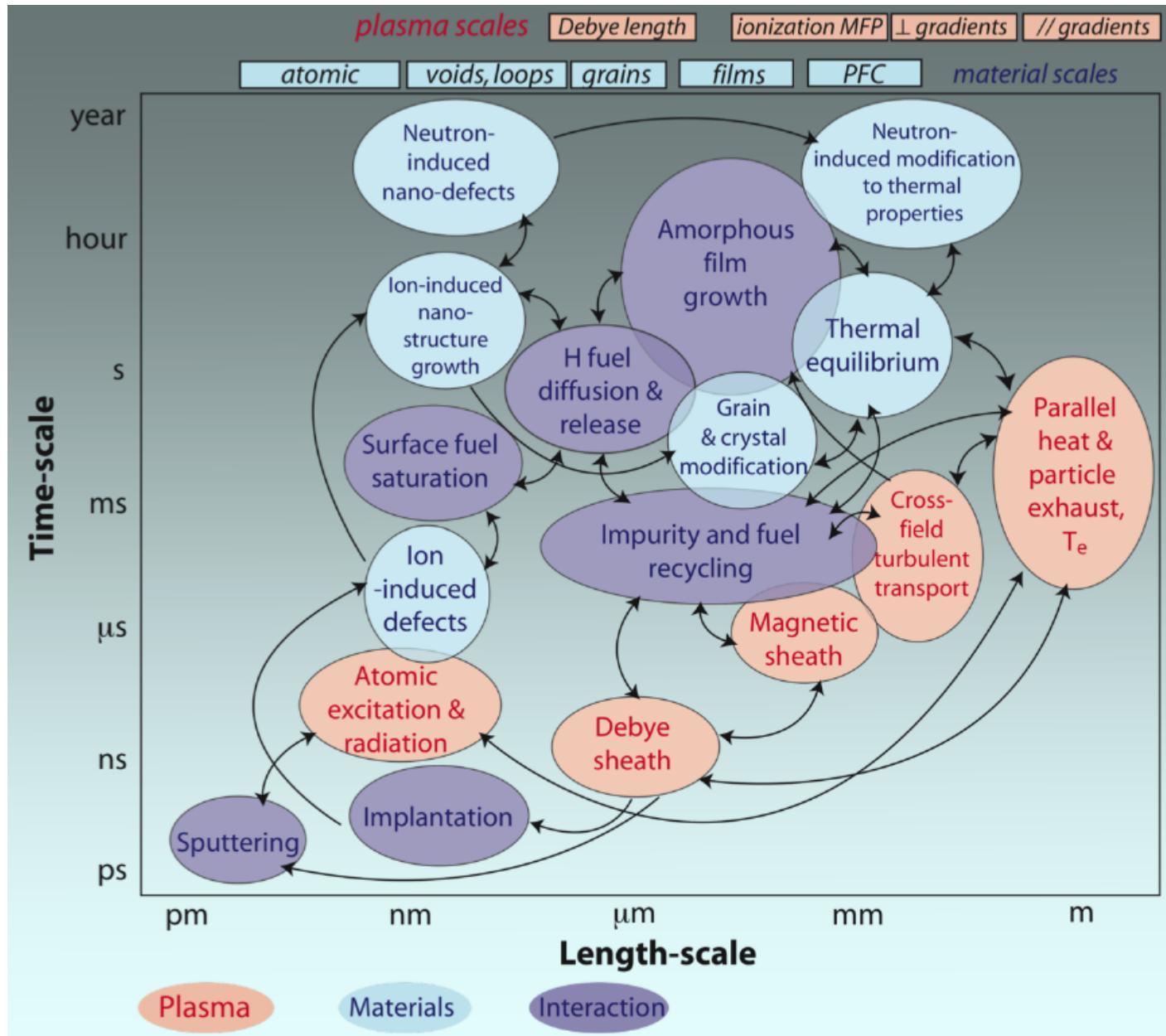
* R.A. Pitts, IHHFC Workshop, Dec 2008

Complex, interlinked PSI phenomena*



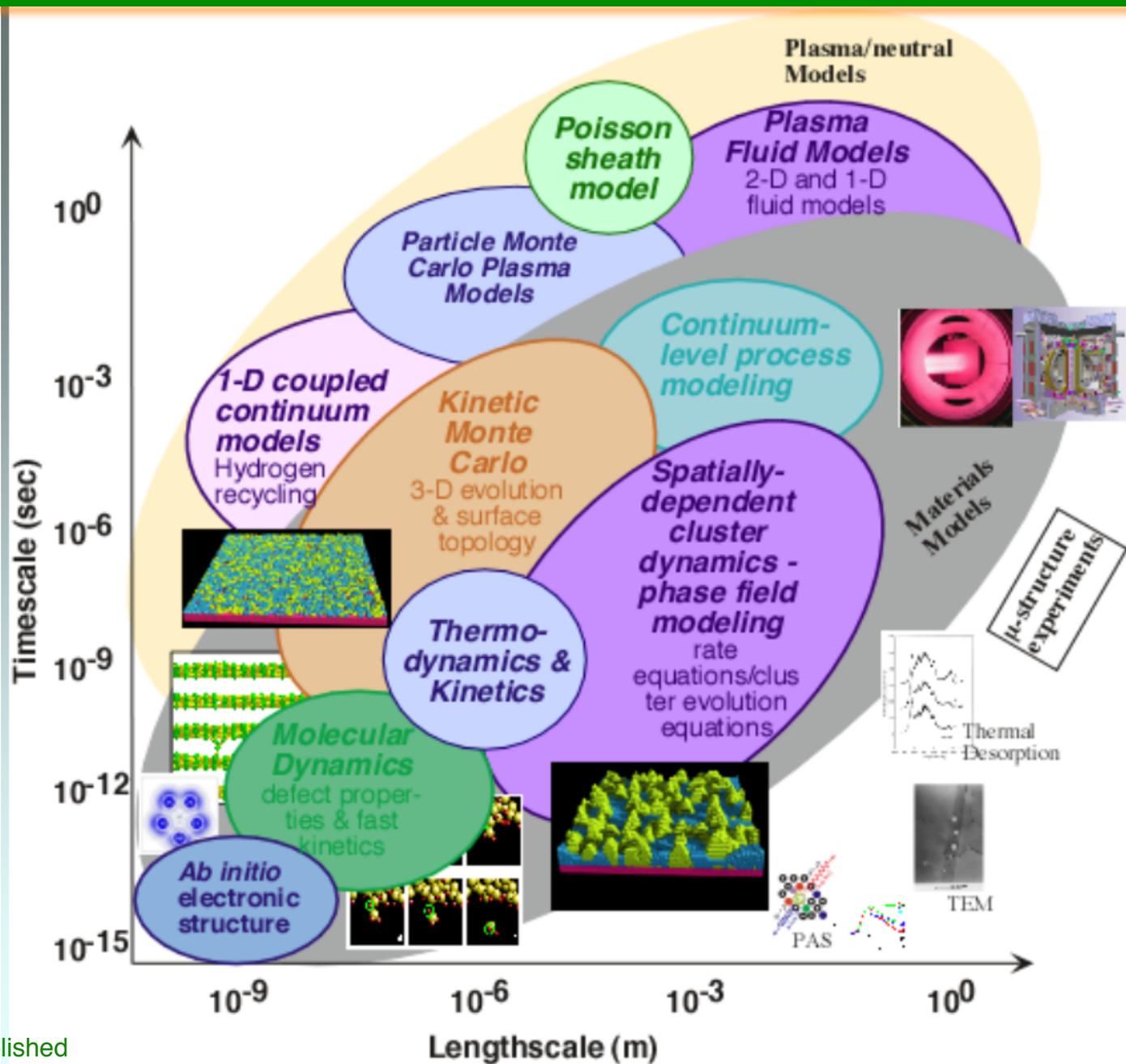
* Whyte and Wirth, unpublished

Multiscale, interlinked Plasma-Surface Interaction phenomena*



* Whyte and Wirth, unpublished

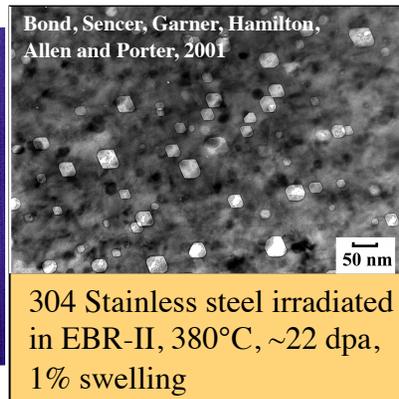
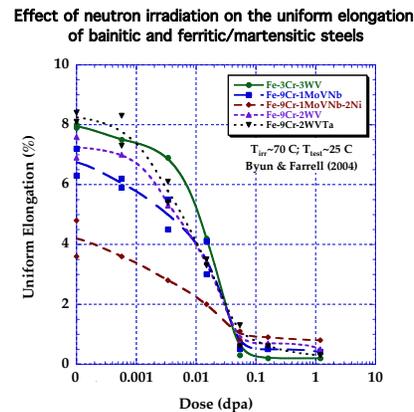
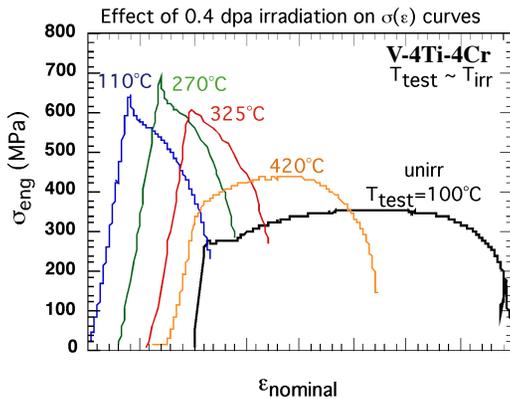
Multiscale modeling capability – a work in progress*



* Whyte & Wirth, unpublished

Irradiation effects on structural materials

- Exposure to neutrons degrades the mechanical performance of structural materials and impacts the economics and safety of current & future fission power plants:
 - Irradiation hardening and embrittlement/decreased uniform elongation ($< 0.4 T_m$)
 - Irradiation ($< 0.45 T_m$) and thermal ($> \sim 0.45 T_m$) creep
 - Volumetric swelling, dimensional instability & growth ($0.3 - 0.6 T_m$)
 - High temperature He embrittlement ($> 0.5 T_m$); **Specific to fusion & spallation accelerators**
- Additional environmental degradation due to corrosive environments (SCC, uniform/shadow corrosion, CRUD)



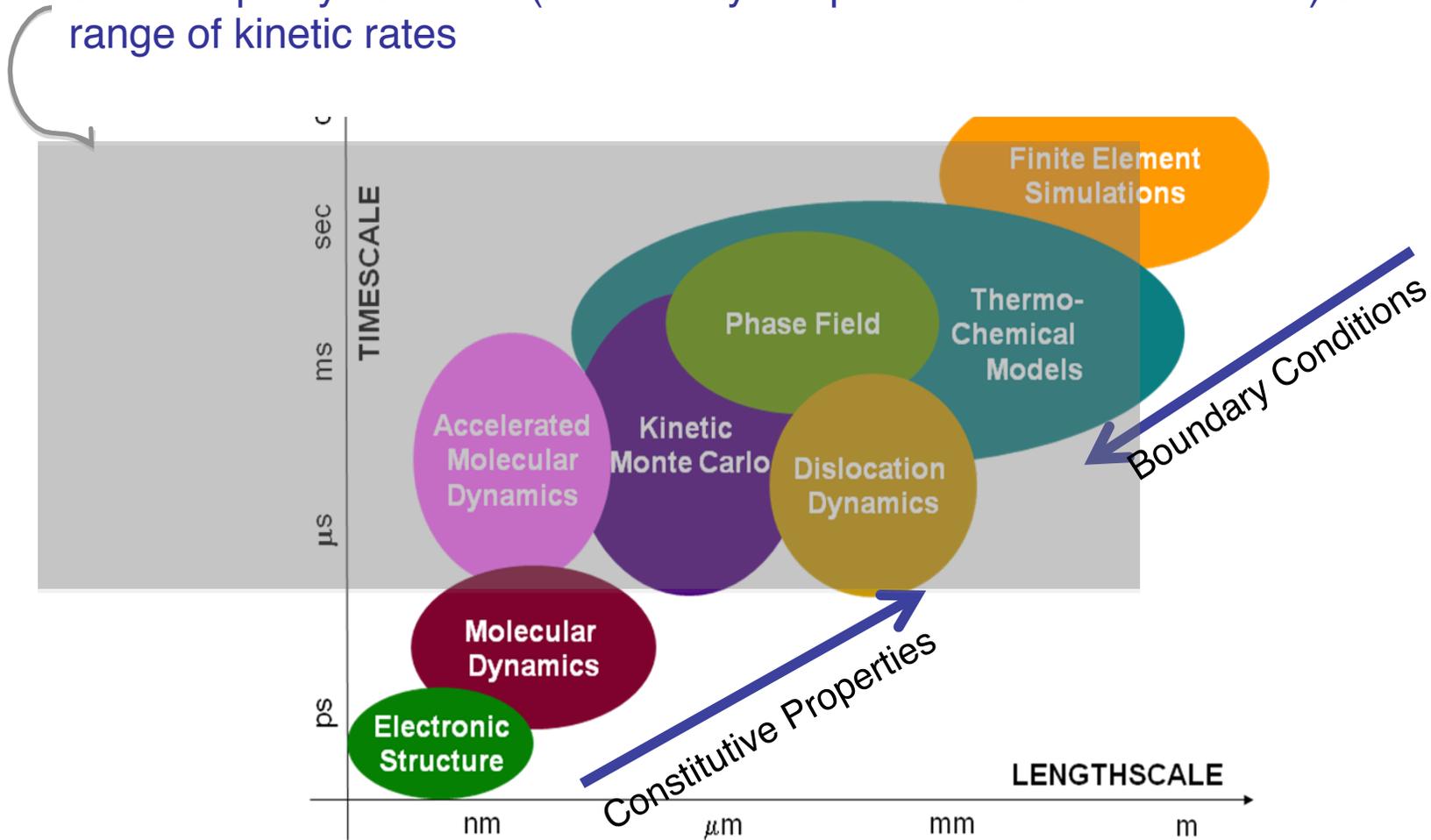
Variables

- Structural Materials (Fe-based steels, Vanadium and Ni-based alloys, Refractory metals & alloys, SiC) and composition
- Zr alloy cladding
- Initial microstructure (cold-worked, annealed)
- Irradiation temperature
- Chemical environment & thermal-mechanical loading
- Neutron flux, fluence and energy spectrum
 - materials test reactor irradiations typically at accelerations of $10^2 - 10^4$

Synergistic Interactions

Multiscale modeling approach – structural materials

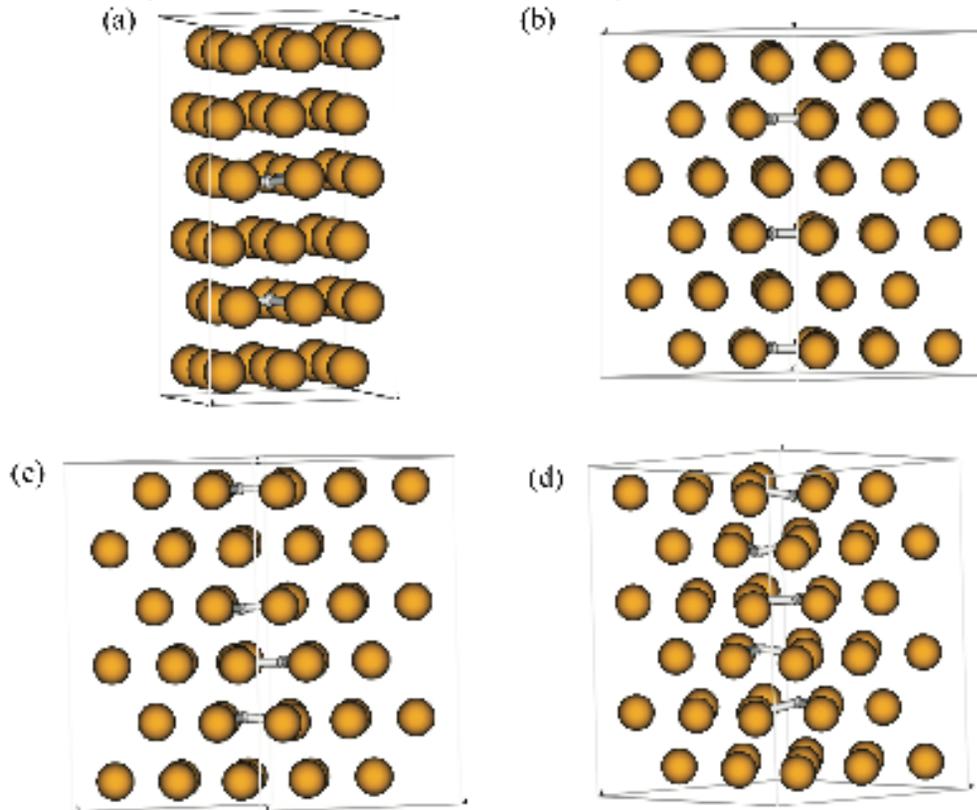
Our biggest scientific challenge is understanding the kinetics of coupled defect – solute/impurity evolution (not entirely unique to irradiation materials) with a wide range of kinetic rates



Electronic structure calculations

- ‘Common’ electronic structure codes: Abinit, Quantum Espresso, VASP

Example – H clusters in Beryllium*



Density Functional Theory applications to investigate structure and energetics of Plasma surface interactions

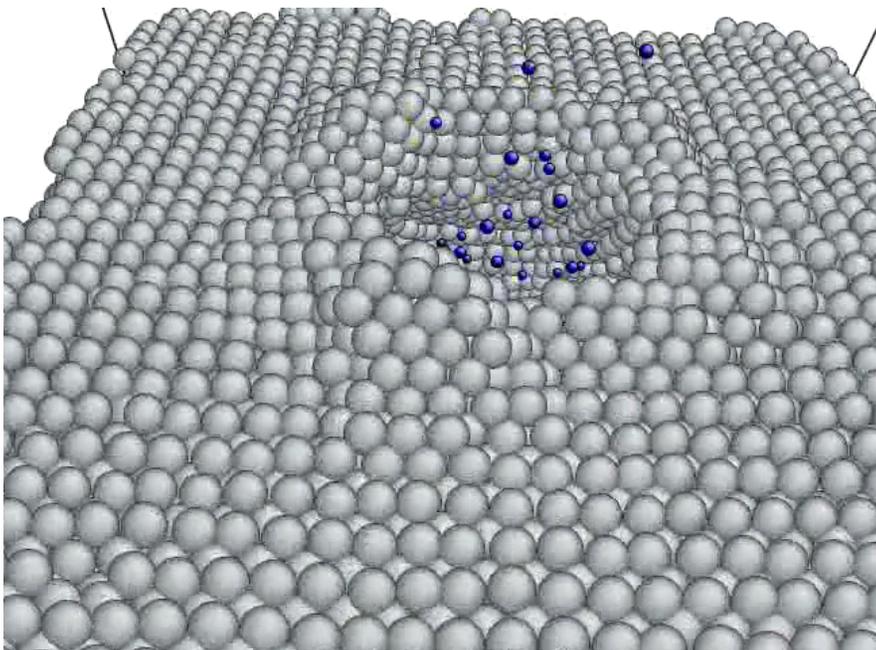
Generally scale well up to 1000's of processors

* A. Allouche, M. Oberkofler, M. Reinelt, and C. Linsmeier, *J. Phys. Chem. C* 114 (2010) 3588-3598.

Molecular Dynamics calculations

- **‘Common’ MD codes: LAMMPS, SPASM**
 - typically run on small, clusters (usually because of throughput), especially for ‘discovery’ science
 - LANL has demonstrated SPASM for 1 billion atoms for 1 nanosecond
- **Accelerated MD codes**
 - LANL demonstrated Parallel Replica Dynamics on 1000 atoms and 12,000 replicas

Bursting of He bubble onto W surface



Road Runner experience (SPASM):

Flop count: petaflop

core-hour per run: 2.8 million

number of cores: 120,000

wall clock time: 24 hours

total memory: 12000 GB

minimum memory per core: 0.1 GB

total data read & written per run: 100 GB

size of checkpoint file: 0.1 GB

Spatially-dependent cluster dynamics model

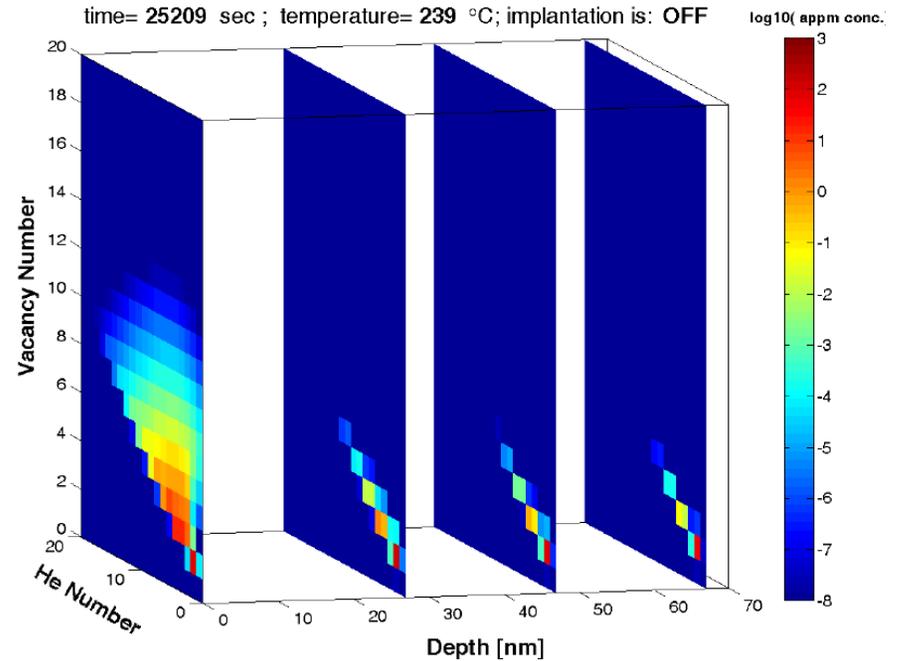
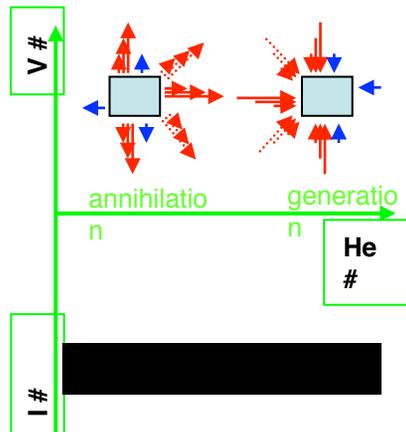
- Dimensionality**

- 1 spatial dim.: x, non-uniform grids
- 1 temporal dim.: t, non-uniform grids
- 1.5 phase-space dims: He#, V(I)#

- What kind of transitions?**

Any cluster can annihilate (transform to another) or be created (transformed from another) :

- **Capturing**: all directions, all step sizes possible, depending on existing mobile species; **including bubble coalescence**
- **Dissociating**: single He, V, I, only



Calculations can involve $> 10^7$ coupled reaction – diffusion differential equations – utilize parallel solvers (PARDISO)

$$\frac{\partial[\text{He}_i]}{\partial t} = D \frac{\partial^2[\text{He}_i]}{\partial x^2} + \text{dissoc.}_{\text{rate}}(\text{He}_m \text{V}_n) + \text{He_kickout_rate}(\text{HeV}) + \text{implan.}_{\text{rate}} - \text{self_trap_rate} - \text{He}_i\text{-trap_rate}(\text{He}_m \text{V}_n) - \text{annihilation by dissociation}$$

PARASPACE Model construction

- **How to describe the rates?**

- capture: $C1 + C2 \rightarrow C3$;

$$R_{+,1,2} = k_{+,1,2} [C1][C2]; \quad k_{+,1,2} = 4\pi(r_1 + r_2)(D_1 + D_2) (\times \text{Bias, if both interstitial type})$$

$$r_{(V_n)} = n^{1/3} r_a$$

$$r_{(I_n)} = \sqrt{\frac{nV_a}{\pi b}}$$

$$D = D_0 \exp(-E_m / k_B T)$$

- dissociation: $C3 \rightarrow C1 + C2$;

$$R_- = k_- [C3]; \quad k_- = k_{+,1,2} C_0 \exp(-E_{b,lin3} / k_B T)$$

- **Boundary conditions (BC)**

black BC, i.e., all concentrations are zero on the surfaces

- **Spatial derivative (finite difference)**

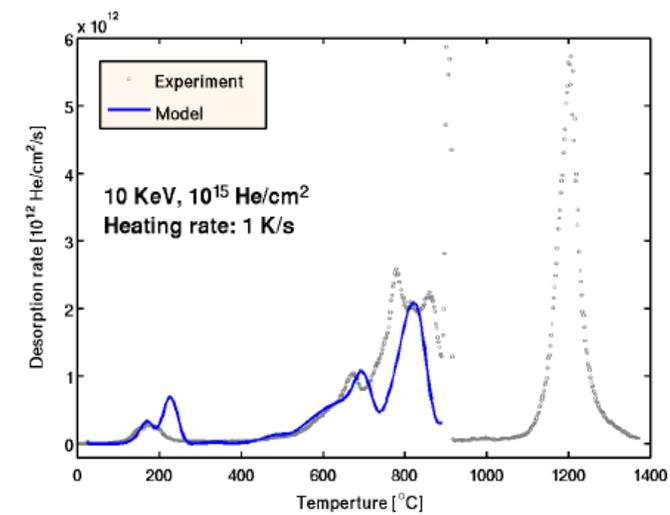
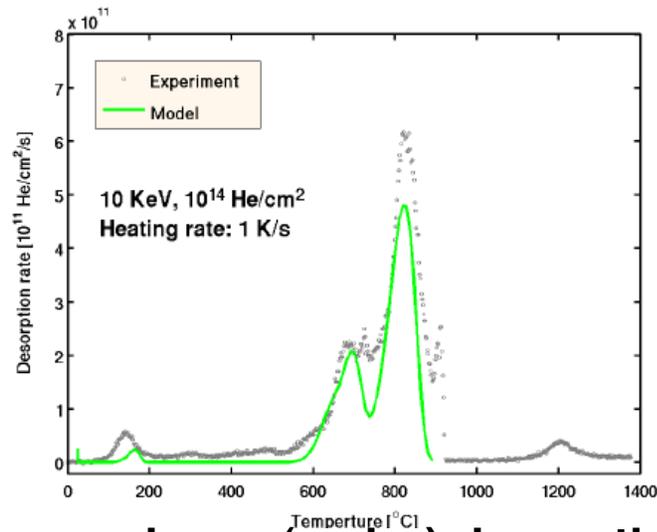
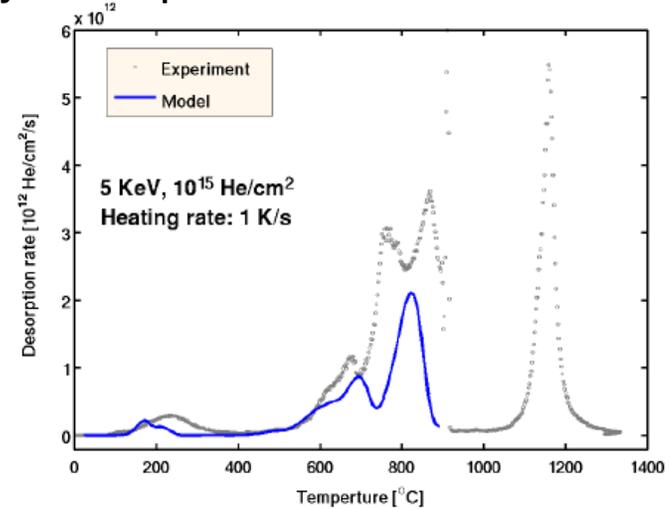
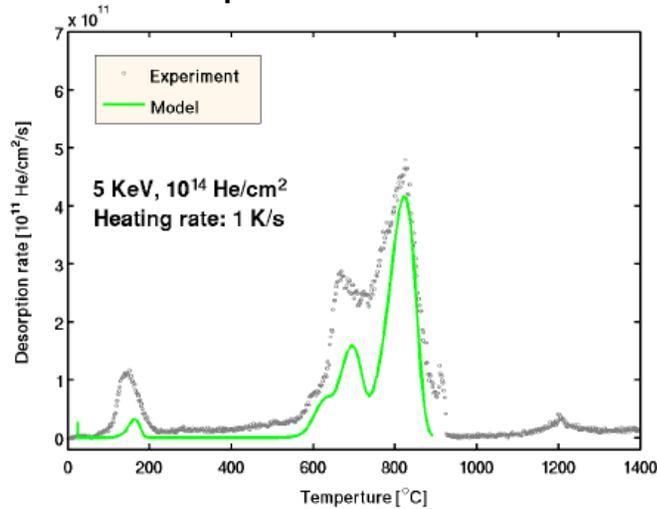
$$\frac{\partial^2 C_i^{x_n}}{\partial x^2} = \frac{(C_i^{x_{n+1}} - C_i^{x_n})}{x_{n+1} - x_n} - \frac{(C_i^{x_n} - C_i^{x_{n-1}})}{x_n - x_{n-1}}$$

$$= \frac{(x_{n+1} - x_{n-1})/2}{(x_{n+1} - x_{n-1})/2}$$

- **Parallel, large sparse-matrix linear solver (PARDISO) using open-MP formalism and backward difference time integration - easily treat systems with 10^7 degrees of freedom**

Spatially-dependent rate-theory based modeling

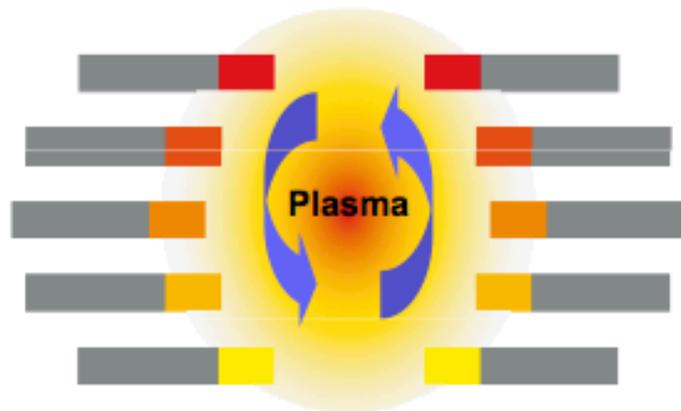
- Thermal desorption behavior of low-energy He implanted into iron



- Model reproduces (major) desorption groups & approximate peak Temp's
- Model overestimates He-leakage during room T relaxation

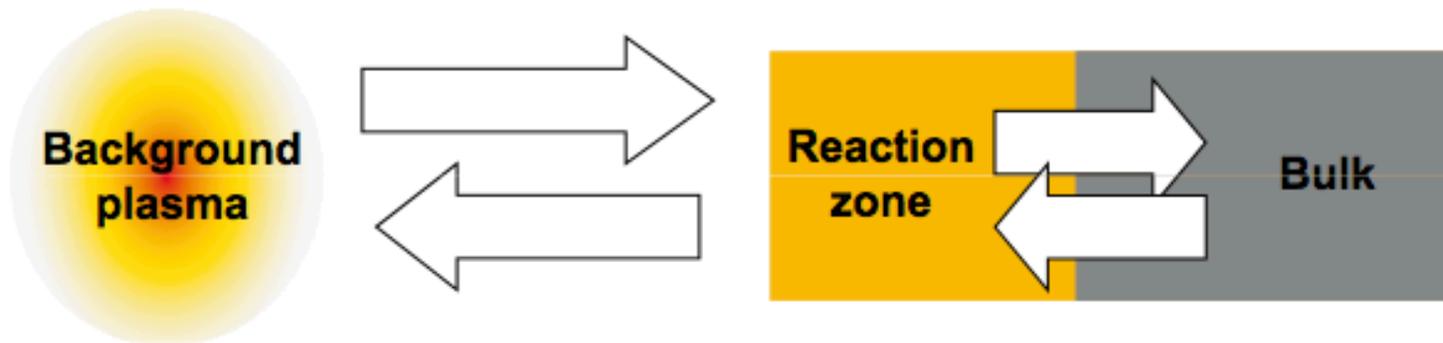
Spatially-dependent reaction-diffusion modeling*

Treat complex **plasma-wall interactions** and **material evolution** in a simplified way



Analytical model:

- first wall: n tiles, different loads
- background plasma (B2 + EIRENE ...)
- redistribution matrix (DIVIMP)
- SDTrim sputter yields
- parametrized surface materials evolution



* C. Linsmeier, PSI-19 invited presentation

Summary & Future Challenges

- **Fusion materials performance is an inherently multiscale challenge – significant effort ongoing to utilize multiscale materials modeling and high performance computing – but this is in the early stages of research and implementation – lots of effort at different scales, few (none) integrated codes using high-performance computing**
- **Key techniques for 1000's of core processing are density functional theory (Abinit, Quantum Espresso, VASP)**
- **Molecular dynamics simulations widely used – but predominately at the individual computing cluster level**
- **Reaction-diffusion solution approaches being developed for defect/surface/chemistry evolution -> will eventually be the large-scale, high performance computing platform to integrate with edge plasma modeling**
- **Monte Carlo approaches are also being pursued – and particle in cell models for the near surface plasma ionization response (LANL VPIC demonstrated at petaflop scales)**
- **Continuing development of knowledge and models through Fusion Simulation Project, etc. leading to increased modeling investigation of Plasma Surface Interactions and Bulk Fusion Materials investigation**